

# Building a Robust Numerical Model for Mass Transport Through Complex Porous Media

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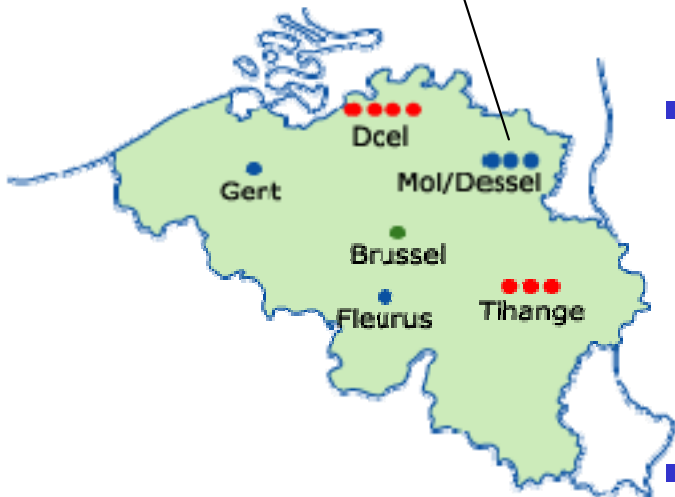
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ONDRAF/NIRAS

# Background

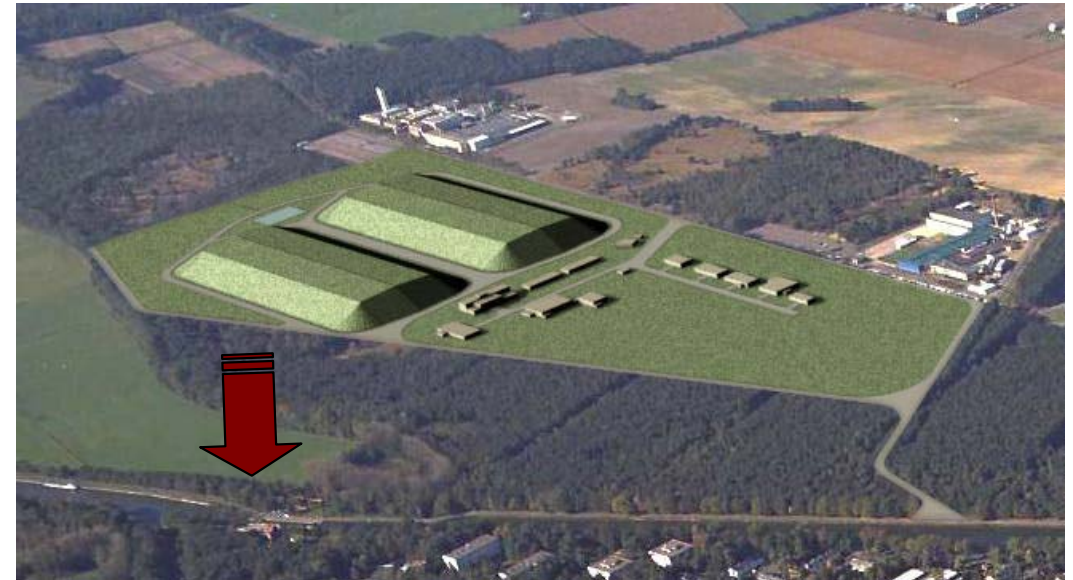
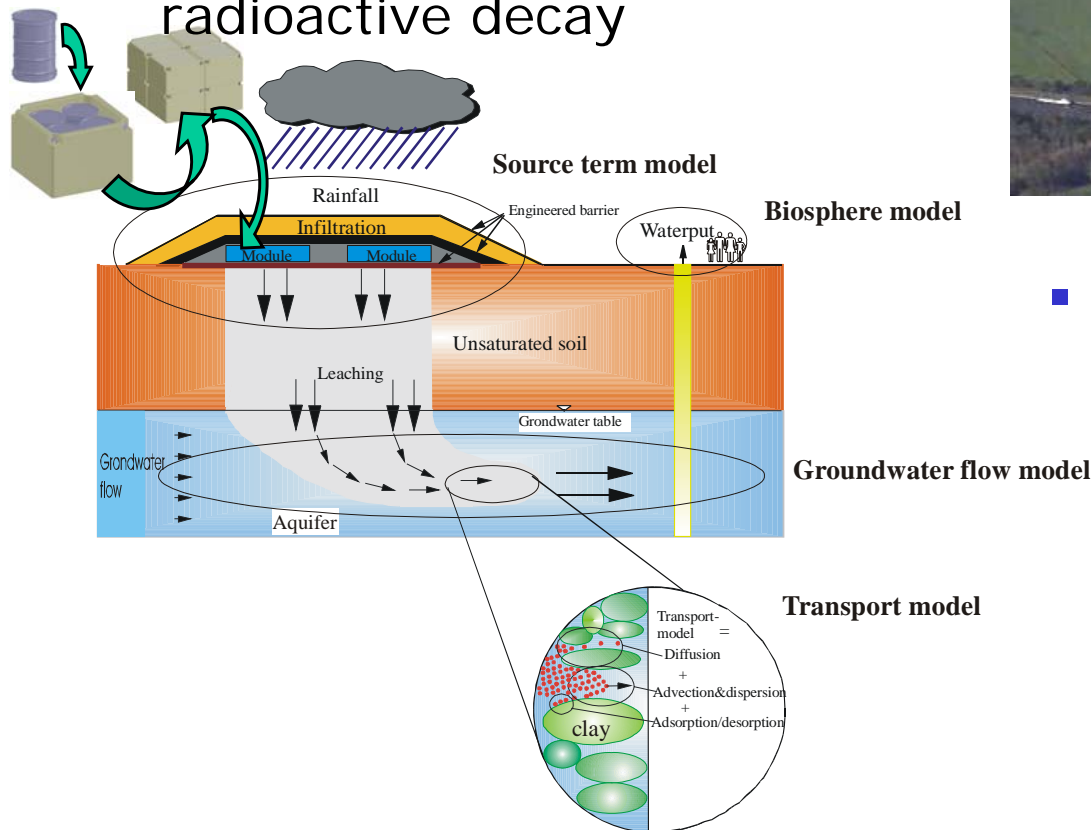
## Sources of radioactive waste in Belgium



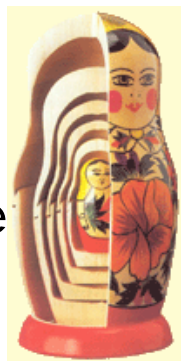
- 7 nuclear power plants (5.5 GWe)
- Nuclear research (SCK•CEN, IRE, ...)
- Nuclear fuel fabrication plants (Belgonucleaire 1986-2006)
- The fabrication of radionuclides and their use in industry, medicine and research (Radium !)
- The dismantling of nuclear facilities (large volumes, relatively low activities)
- LILW-SL:
  - operational LILW-SL 18 700 m<sup>3</sup> (~75% NPP)
  - dismantling LILW-SL 51 800 m<sup>3</sup> (93% concrete)
- HLW: 2100 m<sup>3</sup> (reprocessing)

# Background General

- Long-term safety of surface disposal facility for radioactive waste (LILW-SL)
- The purpose is to isolate radionuclides from the environment >> allow for radioactive decay



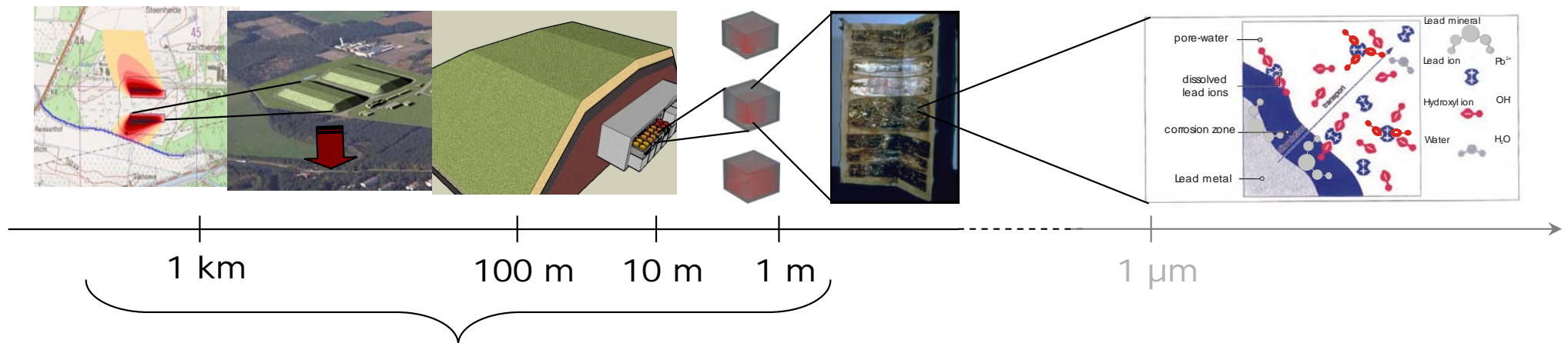
- Modelling radiological impact of radionuclide release from a multi-barrier repository to the geosphere and biosphere (i.e. humans) (Safety assessment)



# Background

## Some related issues

- Enormous **complexity** of interconnected physical-chemical phenomena acting on a wide range of length scales



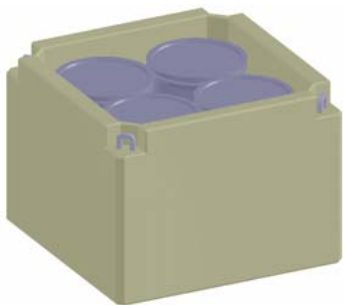
- First simplification is division into three linear related components
  - Near field – transport of radionuclides through engineered barriers (**porous media**)
  - Geosphere – detailed modelling of groundwater flow and radionuclide migration
  - Biosphere – calculation of radiological impact from the use of groundwater for drinking, irrigation of fields and watering cattle

# Background

## Near field



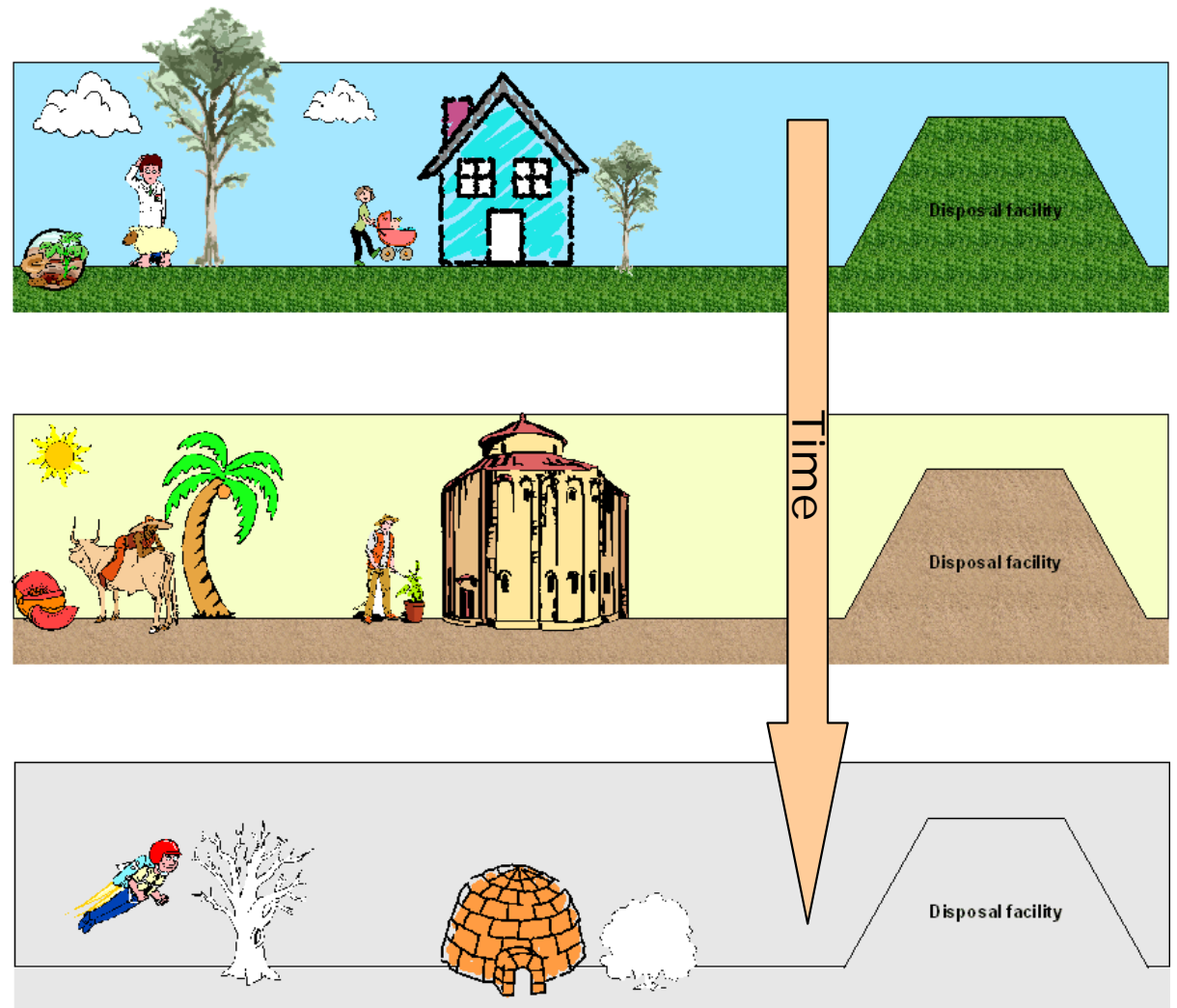
- Near field comprises
  - Multilayer cover (limit water infiltration)
  - Structural components of the vault (module) – e.g. concrete walls, floor, roof
  - Concrete waste disposal containers (monoliths)
    - ♣ Concrete “box” (different dimensions)
    - ♣ Conditioned waste (different dimensions and conditioning) + bulk waste
    - ♣ Backfill grout to fill void spaces between waste and box



# Background Time uncertainty

- Long time frames involved in modelling ( $10^6$  y) which introduces uncertainty about:
  - Human habits
  - Physical parameters
  - Boundary conditions

## Time uncertainty



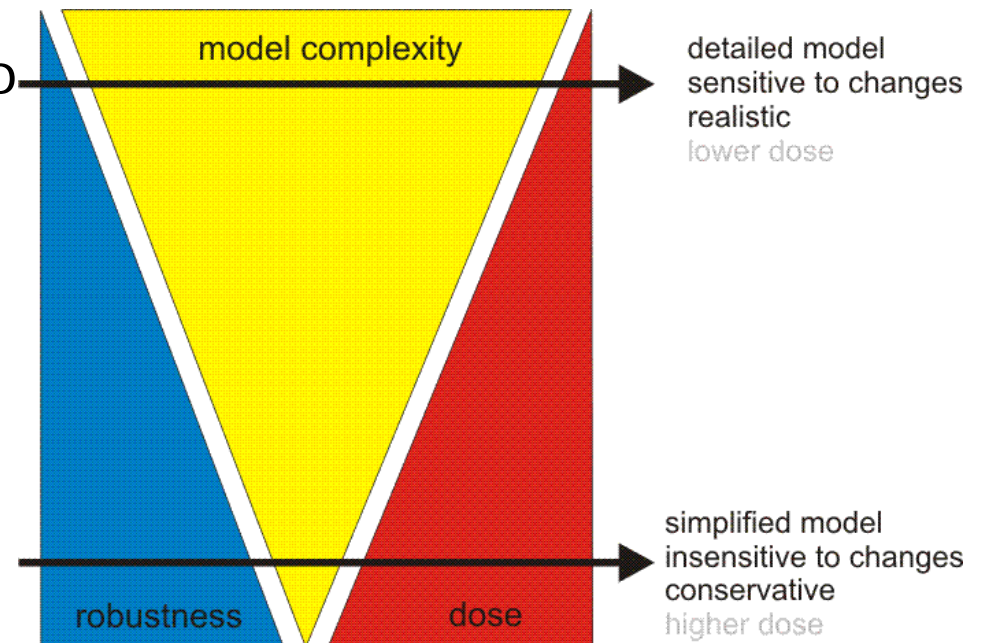
# Background

## Conclusions

- Safety assessment over very long times can only provide qualitative results (order of magnitude estimates)
- Model simplification for dimensions, processes, boundary conditions required due to:

- Large number of calculations
- Too detailed models can be too sensitive
- Lower dimension models are easier to explain
- 3D models are numerically difficult to implement, time consuming and the least stable
- **Any simplification must be proven to be conservative**

**ROBUSTNESS**



# Radionuclide transport modelling

## Numerical challenges

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- Large difference between high initial concentrations in the source zone and low output concentrations (up to several tens of orders of magnitude)
  - Sensitive to numerical error (time and space discretization)
  - Any oscillation can lead to physically unrealistic result (negative concentrations)
- Oscillations may occur because of step function initial conditions, especially for high Pe numbers
- Solutions to avoid oscillations
  - Time and mesh discretization refinement at critical points
  - Smoothing of step functions (*flc2hs* COMSOL function)
  - Logarithmic concentrations → seldom solves the problem
  - Streamline and/or crosswind diffusion



**Easier implemented in 1D than in 3D!**



# Problem description

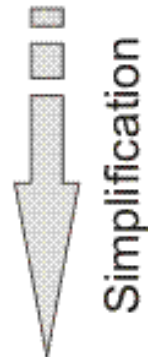
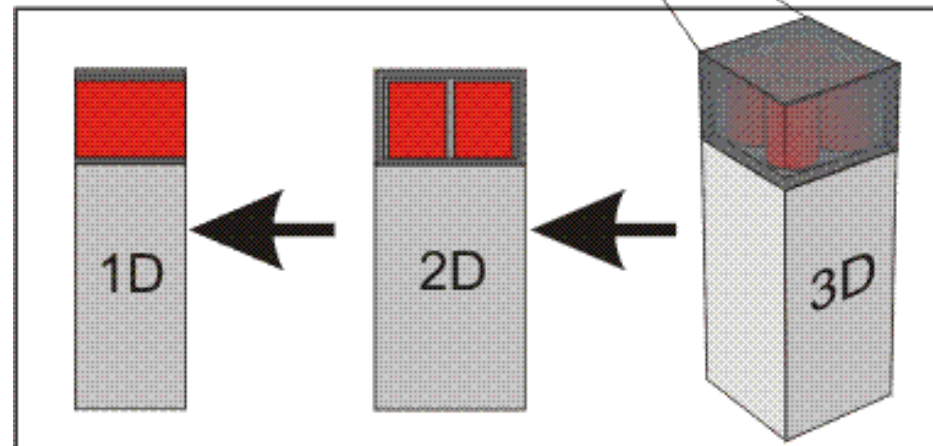
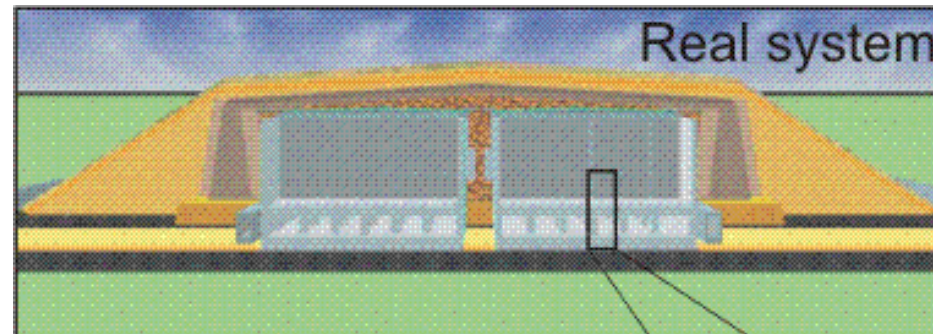
- The aim is to demonstrate the level of simplification still allowed for different contaminant release mechanisms

- Instantaneous release
- Diffusional release
- Dissolutional release



Related to waste conditioning

- Use of COMSOL Multiphysics!



# Physics and modelling approach

## Instantaneous release

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- Instantaneous release assumes all source activity is released instantaneously and completely
- This is the most conservative release model
- Radionuclides are leached from the conditioned waste by infiltrating water and by diffusion
- Imposed uniform water flux across the facility
  - Corresponds to degraded concrete (highly conservative)
  - Concrete retains sorption properties for contaminants
  - Simplifies the migration problem to solute transport equation only
- Comparison of leaching rates in 3D, 2D and 1D for two drum types (220 l and 400 l) and two radionuclides ( $^{36}\text{Cl}$  and  $^{129}\text{I}$ )

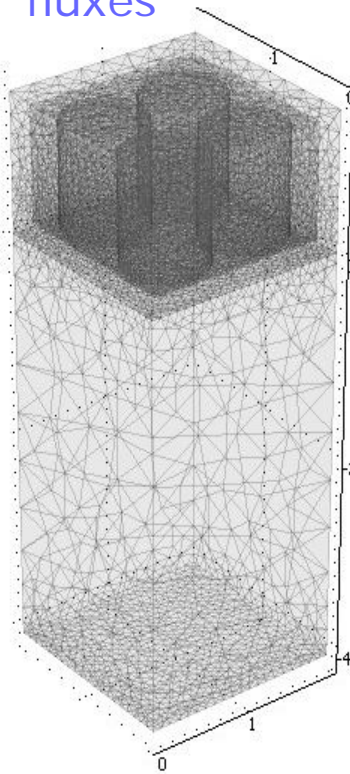
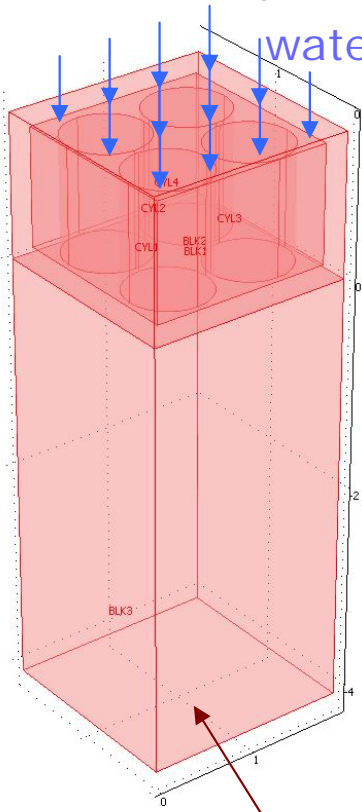
# Physics and modelling approach

## Instantaneous release

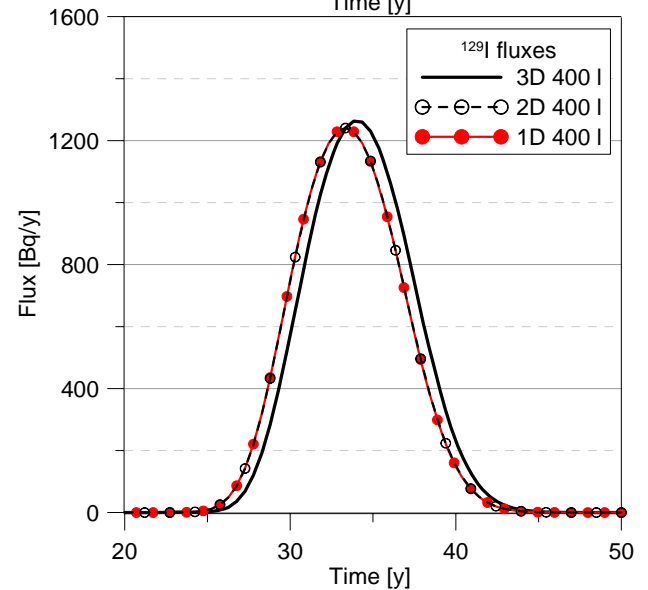
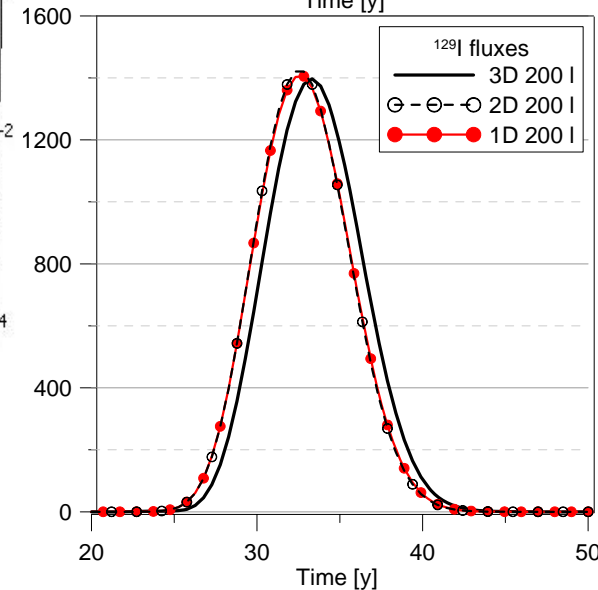
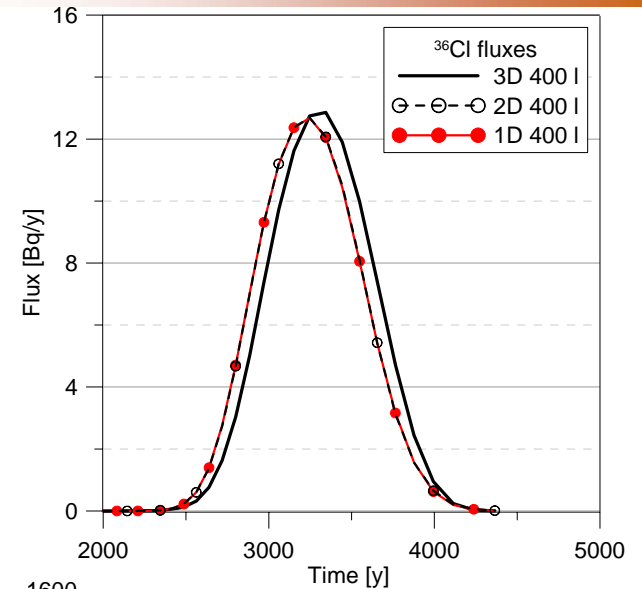
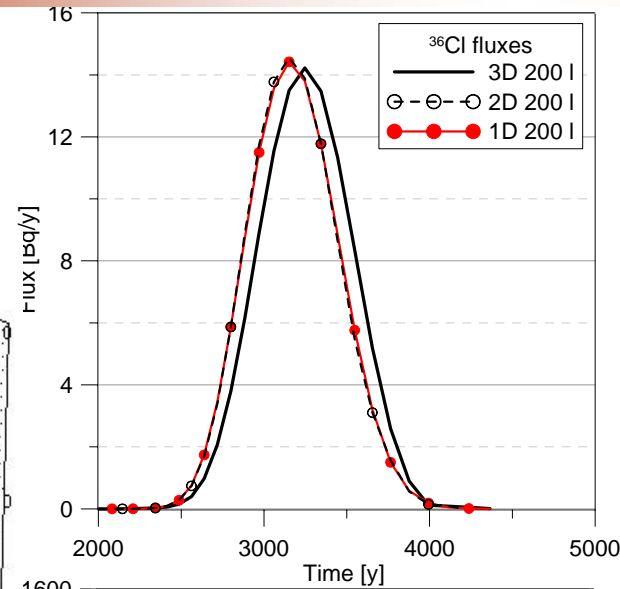
### 3D model example

Geometry

Mesh



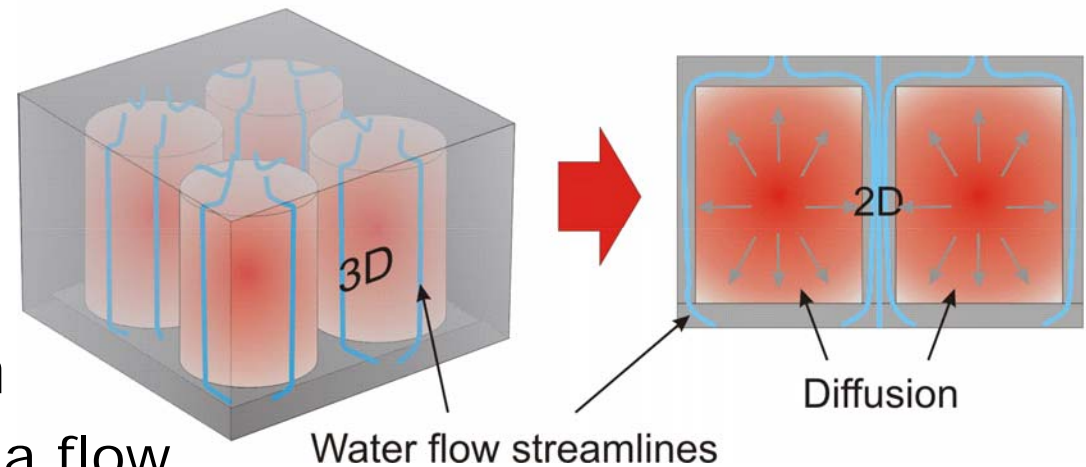
Integrated radionuclide fluxes



# Physics and modelling approach

## Diffusional release

- Diffusional release is the release of radionuclides by diffusion through a porous waste form only
  - e.g. cement-conditioned waste form or bituminized waste
- Water flow through the waste form is considered negligible
  - Water flow flows around the waste form



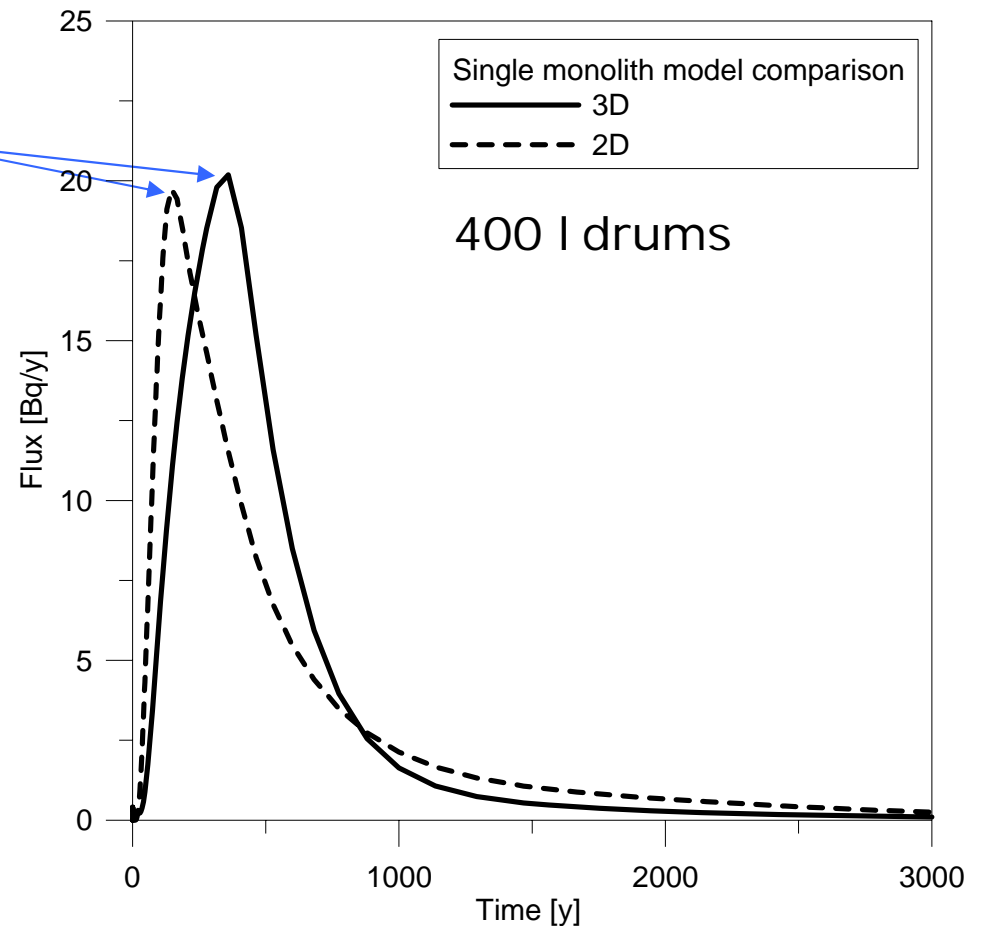
- Decoupled calculation
  - Saturated porous media flow
  - Solute transport (diffusion in waste form, advection outside)

# Physics and modelling approach

## Diffusional release

- Good agreement between 3D and 2D (implementation in 1D is not appropriate)
- Comparison should be made for each packaging type separately

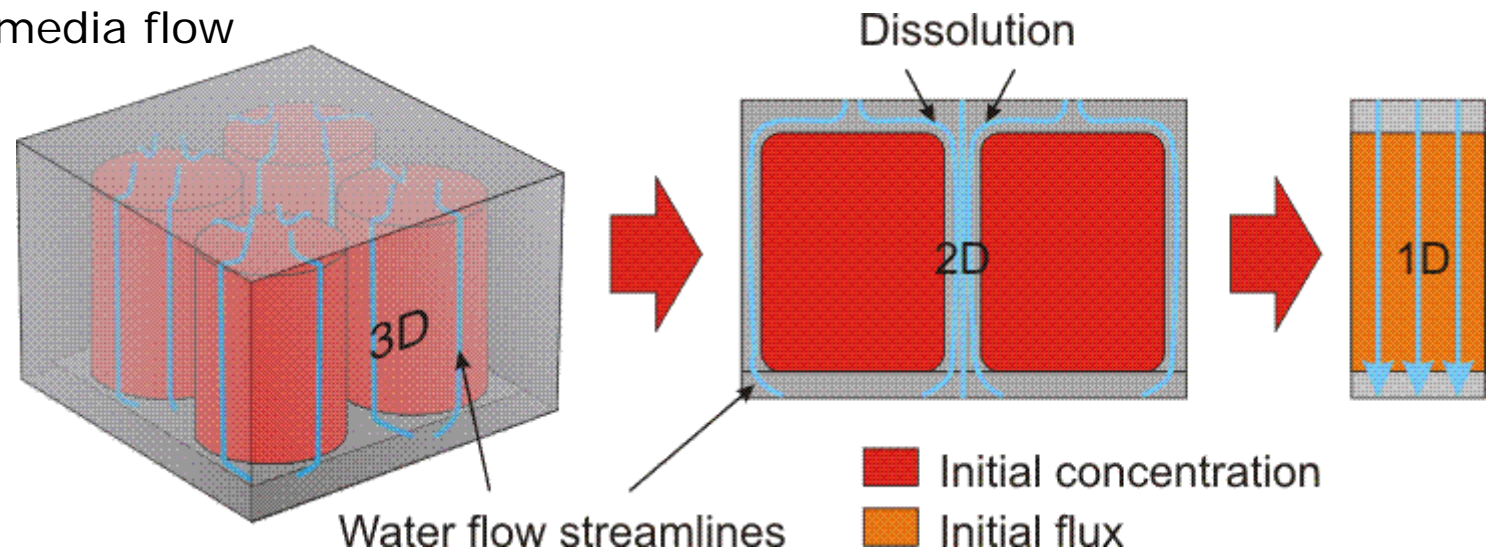
Peak fluxes in good agreement  
Shift in time due to smoothing of source term



# Physics and modelling approach

## Dissolutional release

- Mass release is controlled by the time-dependent dissolution of the waste form
  - e.g. dissolution of vitrified waste, corrosion of steel
- Dissolutional release propagates from the waste form surface inward
- Different source term modeling approach in different dimensions
  - Applied initial concentration in 3D/2D
  - Applied flux in 1D
- Coupled calculation
  - Saturated porous media flow
  - Solute transport



# Physics and modelling approach

## Dissolutional release

- Dissolution defined by dissolution rate  $\delta$  [m/y]
- Due to dissolution, radius  $r(t)$  and height  $z(t)$  of the source and consequently also reactive surface area  $A(t)$  decrease

$$r(t) = \begin{cases} r_0 - \delta t & r_0 \geq \delta \cdot t \\ 0 & r_0 < \delta \cdot t \end{cases} \quad z(t) = \begin{cases} z_0 - \delta t & z_0 \geq \delta \cdot t \\ 0 & z_0 < \delta \cdot t \end{cases} \quad A(t) = 2 \cdot \pi \cdot r(t) (r(t) + z(t))$$

- Smoothed change of hydraulic conductivity

$$e^{\ln k_0 (1 - \text{smooth}(t)) + \ln k_1 \cdot \text{smooth}(t)}$$

$$\text{smooth}_z = \text{flc2hs}(z(t) - z_{\text{bot}}, 0.03) + (1 - \text{flc2hs}(z(t) - z_{\text{top}}, 0.03)) - 1$$

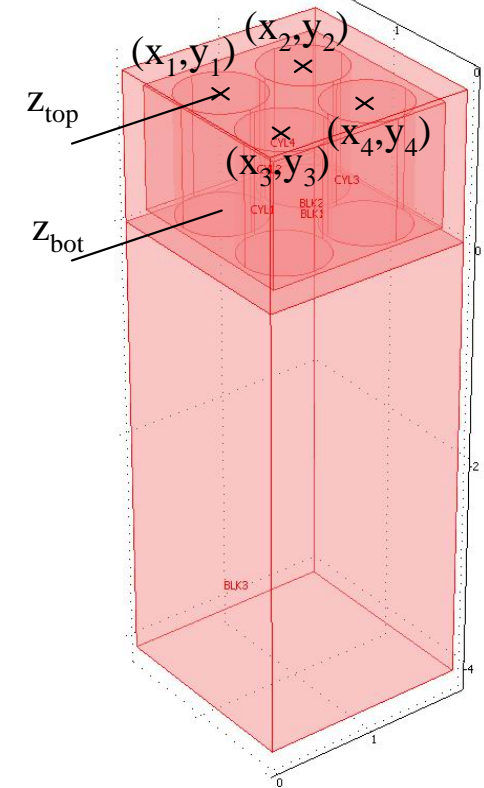
$$\text{smooth}_1 = 1 - \text{flc2hs}(\sqrt{(x(t) - x_1)^2 + (y(t) - y_1)^2} - r, 0.03)$$

$$\text{smooth}_2 = 1 - \text{flc2hs}(\sqrt{(x(t) - x_2)^2 + (y(t) - y_2)^2} - r, 0.03)$$

$$\text{smooth}_3 = 1 - \text{flc2hs}(\sqrt{(x(t) - x_3)^2 + (y(t) - y_3)^2} - r, 0.03)$$

$$\text{smooth}_4 = 1 - \text{flc2hs}(\sqrt{(x - x_4)^2 + (y - y_4)^2} - r, 0.03)$$

$$\text{smooth}(t) = \text{smooth}_z \cdot (\text{smooth}_1 + \text{smooth}_2 + \text{smooth}_3 + \text{smooth}_4)$$



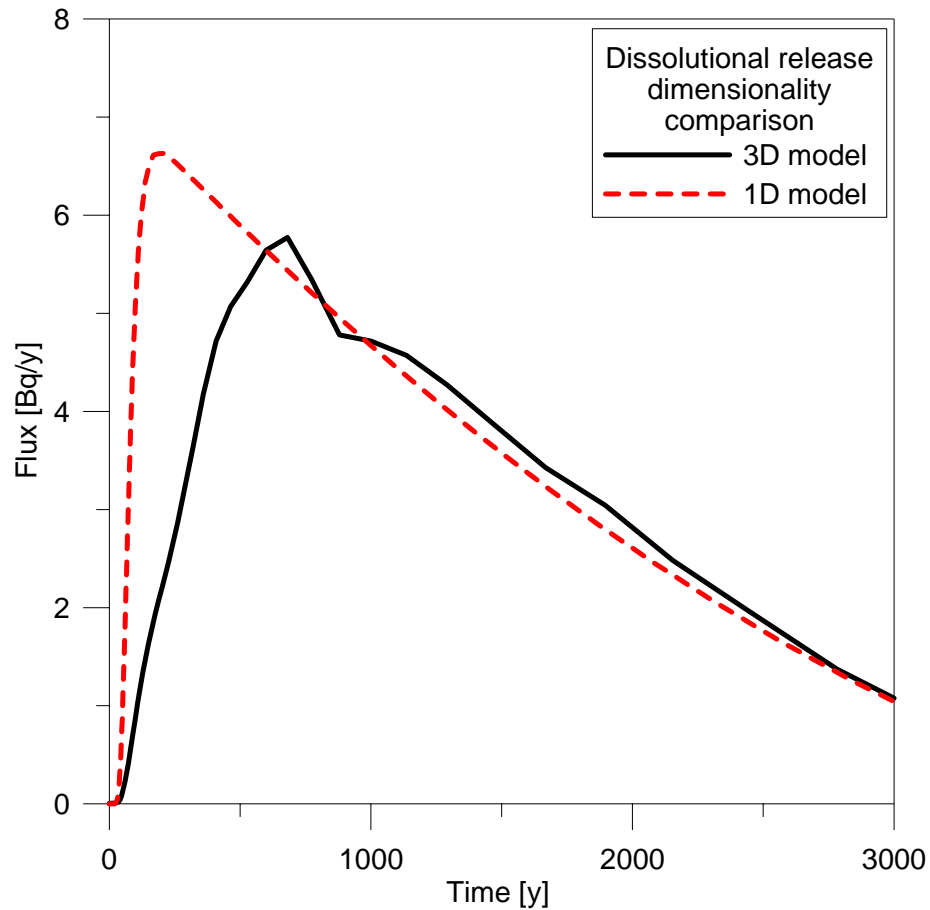
# Physics and modelling approach

## Dissolutional release

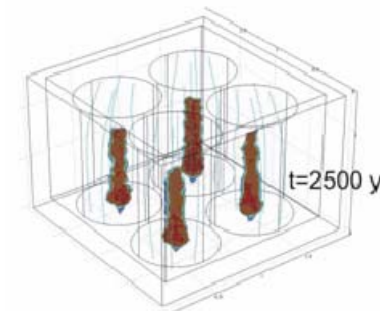
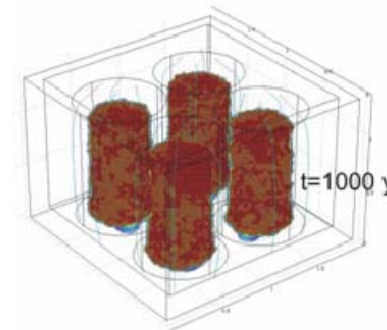
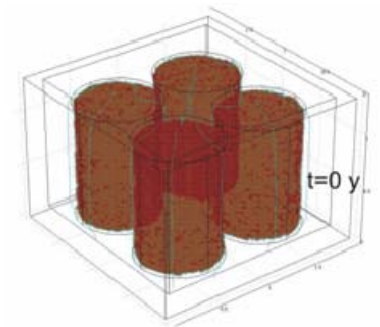
- In 1D the source is defined as radioactive flux

$$F_c [Bq/y] = C_0 \cdot \delta \cdot A(t) \cdot e^{-\ln(2)/T_{1/2} \cdot t}$$

$T_{1/2}$ ....half life



3D results



Shrinking source term



# Conclusions

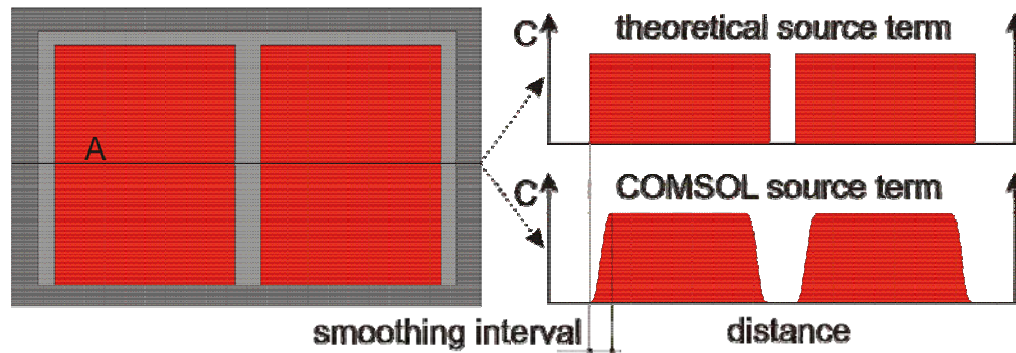
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- Dimensionality reduction should be verified for each different conceptual model
- Sometimes simplification to 1D is not possible (diffusional release)
- For dissolutional release 3D calculations are time consuming
  - less than minute for 1D and 19 hours for 3D calculation
- Instantaneous release model (as defined here) gives equal results in 1D and in 3D
- COMSOL Multiphysics proved to be efficient and versatile tool for safety assessment calculations

# Physics and modelling approach

## Source term

- The problem involves solute transport of decaying and sorbing substances (i.e. radionuclides) in saturated porous media
- Initial condition: concentration in porewater  $C(t_0) = \frac{A}{V \cdot R \cdot \eta}$
- Smoothing applied



- Volume correction due to smoothing  $V = \int_V smooth \, dV$
- Concentration with smoothing  $C(\vec{x}, t_0) = smooth \cdot C(t_0)$